

# 福島原発事故について

## Accident in Fukushima Daiichi NPP

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本資料は、英国SERCO社、ベルギーSCK・CEN研究所、及び仏国CEAとの技術会議において情報提供のみのためにJNES(中江)が作成したものである。

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# Epicenter of the Earthquake



- The earthquake was occurred **at 14:46 on March 11, 2011** in Tohoku district where is northern part of Japan.
- **Magnitude was 9.0 Mw.**
- Epicenter location is **38° 6'' north latitude and 142° 51'' east longitude, and the depth is 24 km.**

source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Tsunami After the Earthquake



- East coast of northern areas in main island of Japan was seriously damaged due to **tsunami**.
- As of 24 August, 15,729 peoples are dead and 4,539 people are missing.

source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Nuclear Reactors Near Epicenter of the Earthquake

## Location of the Nuclear Installations



Onagawa	<ul style="list-style-type: none"> <li>Unit1: 524 MW, 1984-</li> <li>Unit2: 825 MW, 1995-</li> <li>Unit3: 825 MW, 2002-</li> </ul>
Fukushima I	<ul style="list-style-type: none"> <li>Unit1: 460 MW, 1971-</li> <li>Unit2: 784 MW, 1974-</li> <li>Unit3: 784 MW, 1976-</li> <li>Unit4: 784 MW, 1978-</li> <li>Unit5: 784 MW, 1978-</li> <li>Unit6: 1,100 MW, 1979-</li> </ul>
Fukushima II	<ul style="list-style-type: none"> <li>Unit1: 1,100 MW, 1982-</li> <li>Unit2: 1,100 MW, 1984-</li> <li>Unit3: 1,100 MW, 1985-</li> <li>Unit4: 1,100 MW, 1987-</li> </ul>
Tokai II	(1,100 MW, 1978-)

source : [www.meti.go.jp](http://www.meti.go.jp)

# Effect of the Earthquake and Tsunami on NPPs

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- **11 NPPs were automatically shut down.**
  - Onagawa Unit 1, 2, 3
  - 1F Unit 1, 2, 3
  - 2F Unit 1, 2, 3, 4
  - Tokai Daini

After the automatic shut down, the units of 1-3 at Onagawa NPS, the units 1-4 at 2F have been cold shut down safely. **However, the units 1-3 at 1F have been failed to get cold shut down condition.**

- **3 NPPs were under periodic inspection.**
  - 1F Unit 4, 5, 6

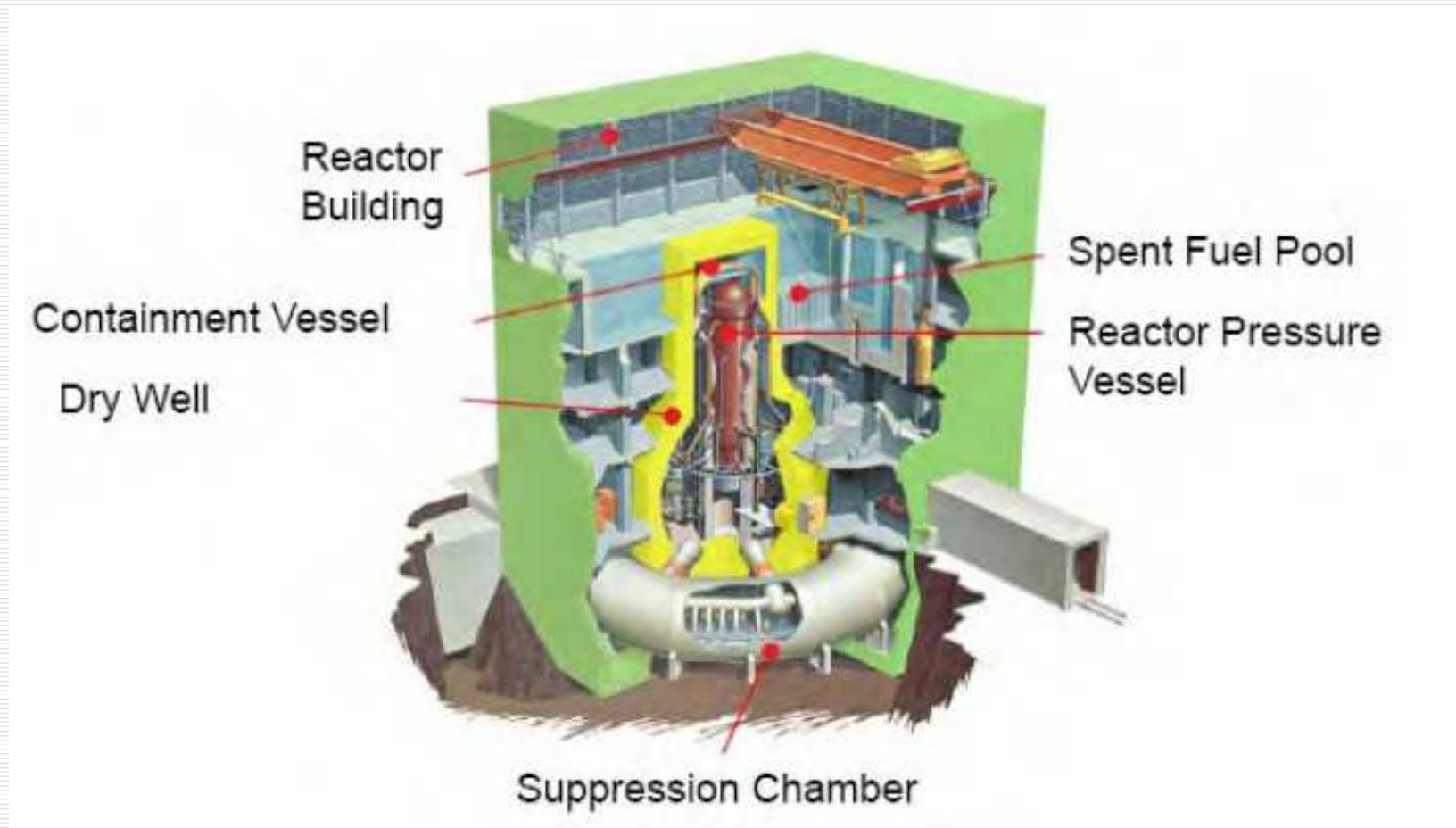
Spent fuels stored in the SFP at the units 5 and 6 have been cooled safely. **However, those at unit 4 have not been cooled.**

# Location of Fukushima Daiichi (1F) NPPs



source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Overview of Mark-I Type BWR (1F1-4)



source :USNRC Technical Training Center



# Main Parameters of 1F NPPs

	Unit 1	Unit 2	Unit 3	Unit 4	Unit 5	Unit 6
Type	BWR-3	BWR-4	BWR-4	BWR-4	BWR-4	BWR-5
Containment Vessel (CV) Model	Mark-1	Mark-1	Mark-1	Mark-1	Mark-1	Mark-2
Electric Output (MWe)	460	784	784	784	784	1100
Max. Pressure of RPV (MPa)	8.24	8.24	8.24	8.24	8.62	8.62
Max. Temperature of RPV (°C)	300	300	300	300	302	302
Max. Pressure of CV (MPa)	0.43	0.38	0.38	0.38	0.38	0.28
Max. Temperature of CV (°C)	140	140	140	140	138	171 (D/W) 105 (S/C)
Fuel Type Loaded in the Core	8x8, 9x9	9x9	9x9	9x9	9x9	9x9
No. of Fuel Assembly Loaded	400	548	548*		548	764
No. of Fuel Assembly in SFP	392	615	566	1535	994	940
Commercial Operation	03/1971	07/1974	03/1976	10/1978	04/1978	10/1979
No. of Emergency DG	2	2	2	2	2	3**

\* Thirty two (32) fuel assemblies are MOX fuel.

\*\* One emergency DG is air-cooled.

source : [Application document of license for establishment of NPP](#)

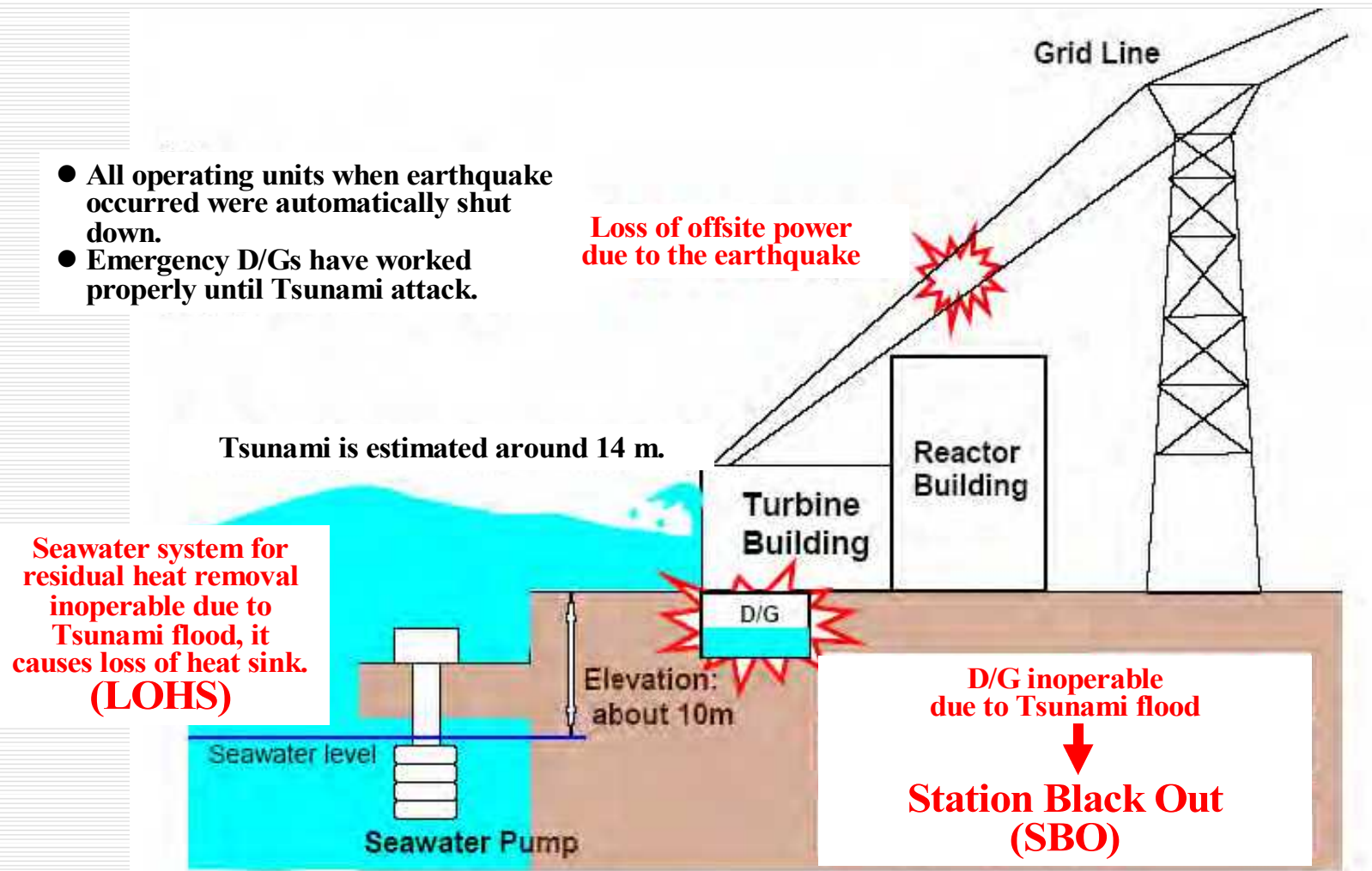
# Plant Status of 1F NPPs Just Before Accident

Unit 1	Unit 2	Unit 3	Unit 4	Unit 5	Unit 6
<b>in Operation</b>	<b>in Operation</b>	<b>in Operation</b>	<b>Refueling Outage</b>	<b>Refueling Outage</b>	<b>Refueling Outage</b>
<b>460MWe</b>	<b>784MWe</b>	<b>784MWe</b>	<b>0MWe</b>	<b>0MWe</b>	<b>0MWe</b>
<b>Spent Fuel Pool 1</b>	<b>Spent Fuel Pool 2</b>	<b>Spent Fuel Pool 3</b>	<b>Spent Fuel Pool 4</b>	<b>Spent Fuel Pool 5</b>	<b>Spent Fuel Pool 6</b>
<b>in normal Operation</b>	<b>in normal Operation</b>	<b>in normal Operation</b>	<b>All fuel assemblies in the R/C were transferred to SFP.</b>	<b>in normal Operation</b>	<b>in normal Operation</b>

source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Effects caused by the Earthquake and Tsunami

- All operating units when earthquake occurred were automatically shut down.
- Emergency D/Gs have worked properly until Tsunami attack.



source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Summary of the Accident with Units 1-3 at 1F

	<b>1F1</b>	<b>1F2</b>	<b>1F3</b>
<b>SBO</b>	<b>15:37 (Mar.11)</b>	<b>15:41 (Mar.11)</b>	<b>15:38 (Mar.11)</b>
<b>LOHS</b>	<b>same as above</b>	<b>same as above</b>	<b>same as above</b>
<b>Water Injection into Reactor Core</b>	<b>05:46 (Mar.12) freshwater</b>	<b>19:54 (Mar.14) seawater</b>	<b>13:12 (Mar.13) seawater</b>
<b>C/V Vent</b>	<b>14:30 (Mar.12)</b>	<b>try but fail</b>	<b>8:41 (Mar.13)</b>
<b>Hydrogen Explosion</b>	<b>15:36 (Mar.12)</b>	<b>after 6:00(Mar.15) at S/C</b>	<b>11:01 (Mar.14)</b>
<b>Fuel Melting</b>	<b>about 17:00 (Mar.11) 100%</b>	<b>about 18:00 (Mar.14) 100%</b>	<b>about 8:00 (Mar.13) 50%</b>

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety

# Causes of the Different Behavior in Units 1-3

	<b>1F1</b>	<b>1F2</b>	<b>1F3</b>
<b>Hydrogen Explosion</b>	<b>15:36 (Mar.12)</b>	<b>after 6:00 (Mar.15) at S/C</b>	<b>11:01 (Mar.14)</b>
<b>Fuel Melting</b>	<b>about 17:00 (Mar.11) 100%</b>	<b>about 18:00 (Mar.14) 100%</b>	<b>about 8:00 (Mar.13) 50%</b>
<b>Kinds of ECCS</b>	<b>IC</b>	<b>RCIC</b>	<b>RCIC HPCI</b>
<b>Operational State of ECCS</b>	<b>Startup of IC at 14:52 Mar. 11</b>	<b>Startup of RCIC at 14:50 Mar. 11 Stop of RCIC at 13:25 Mar.14</b>	<b>Startup of RCIC at 15:05 Mar. 11 Stop of RCIC at 11:36 Mar. 12 Startup of HPCI at 12:35 Mar. 12 Stop of HPCI at 02:42 Mar. 13</b>
<b>Ventilation of R/B</b>	<b>not be functioned</b>	<b>blowout panel broken due to explosion at 1F3</b>	<b>not be functioned</b>

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety

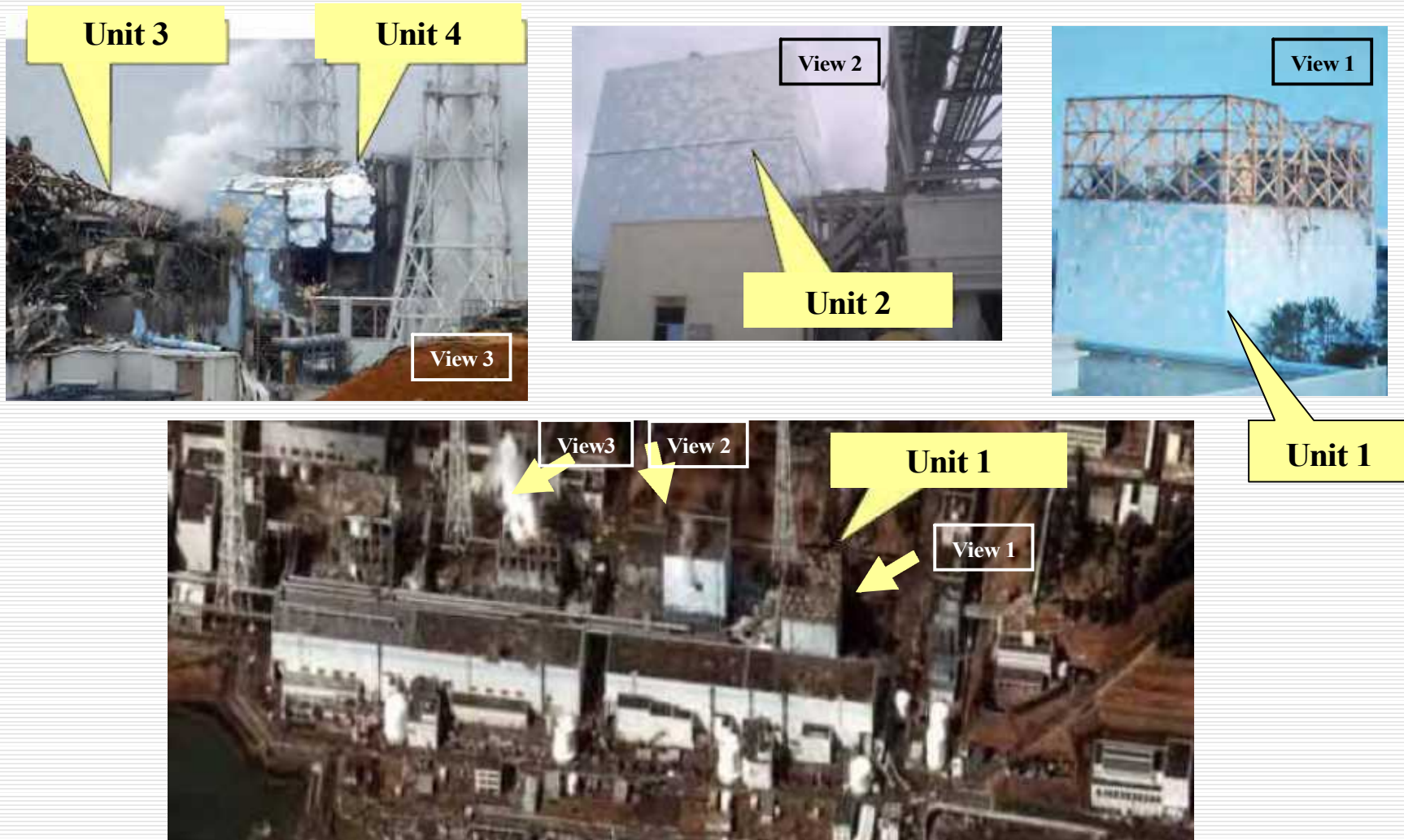
# Summary of the Accident with Unit 4 at 1F

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<b>Spent Fuel Pool in 1F4</b>		
<b>SBO</b>	<b>15:38 (Mar.11)</b>	<b>Spent fuel pool cooling system in 1F4 did not work due to SBO. Vaporization of SFP water occurred and water level decreased. Water may be supplied from the reactor core through cannal.</b>
<b>Hydrogen Explosion</b>	<b>about 6:00 (Mar.15)</b>	<b>Hydrogen production is not taken in 1F4, and it may come from 1F3.</b>
<b>Fuel Failure</b>	<b>Failure?</b>	<b>Fuel assemblies stored in the SFP were observed, also the activity of water in the pool was analyzed. Fuel failure may not be occurred.</b>

source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Photograph of 1F1-4 After Exposure



source : [www.tepco.co.jp](http://www.tepco.co.jp) and [www.digitalglobe.com](http://www.digitalglobe.com)

# Water Discharge by Concrete Pumping Vehicle

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source : [www.tepco.co.jp](http://www.tepco.co.jp)



# Why Severe Accident Occurred?

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## Direct Cause

- Tsunami continued by earthquake caused SBO and LOHS.
- SBO and LOHS are direct causes of severe accident.

## Design and Construction

- The countermeasure against tsunami attack was not enough to prevent SBO and LOHS.
- There are flaws in the guideline of safety design, especially on SBO.

## PSA and Stress Test

- PSA or PRA has been actively performed, but the scope is not enough to cover wide range of accident scenario.
- Stress test has not been done.

## Mind and Education

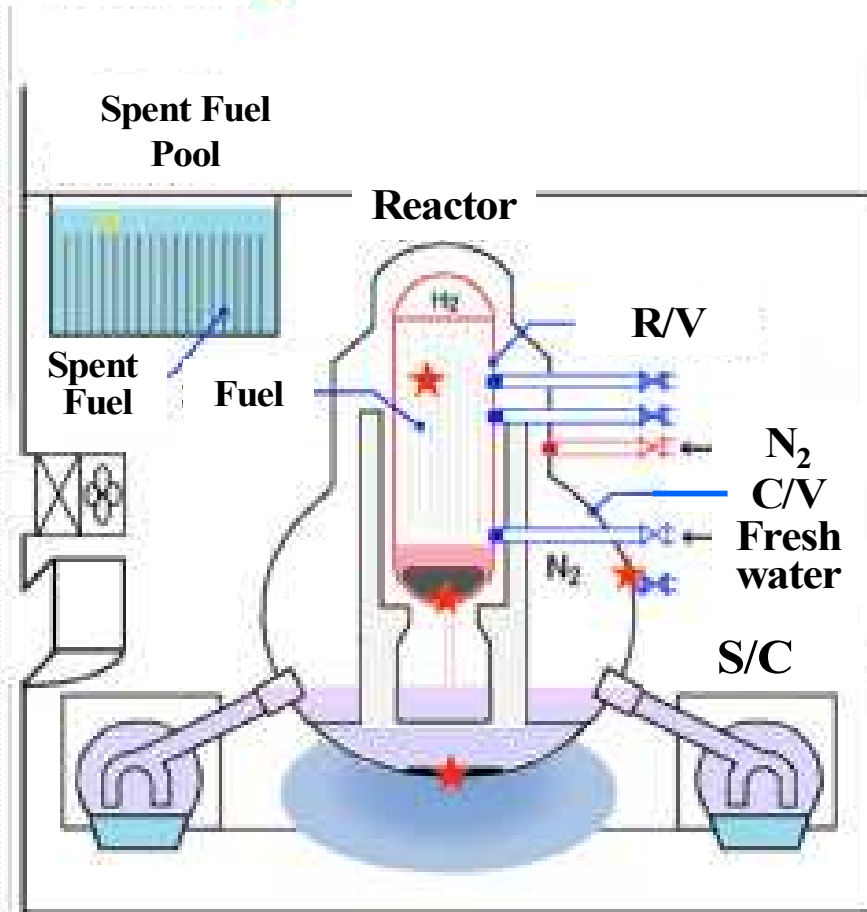
- The occurrence of real severe accident is not in the right mind frame by persons concerned with nuclear engineering.
- The education of operators is not enough against protection of severe accident.

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety

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# Status of 1F1 As of 24 August 2011

Reactor Building

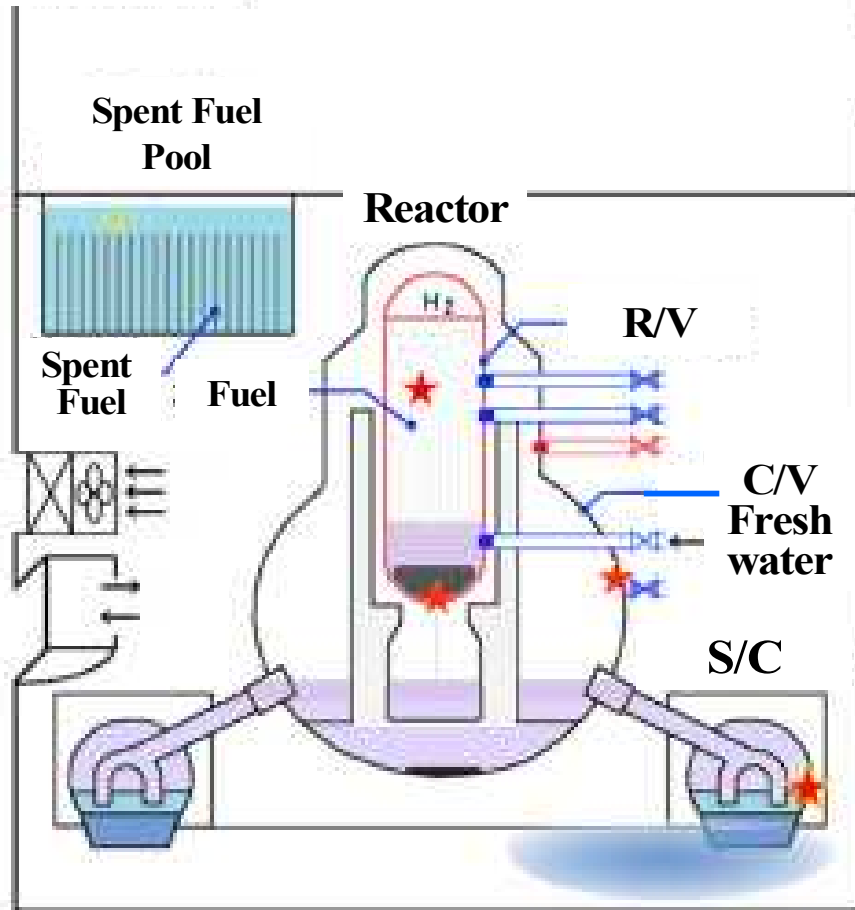


Plant Parameters	1F1
Temperature at feed water nozzle (°C)	91.2
Reactor Pressure (MPa g)	0.017
Reactor Coolant Level (mm)	down scale
Temperature in Spent Fuel Pool (°C)	29.0

source : [www.tepco.co.jp](http://www.tepco.co.jp) and [fukumitsu.xii.jp/syu\\_f/FukushimaGenpatsu\\_1.html](http://fukumitsu.xii.jp/syu_f/FukushimaGenpatsu_1.html)

# Status of 1F2 As of 24 August 2011

Reactor Building

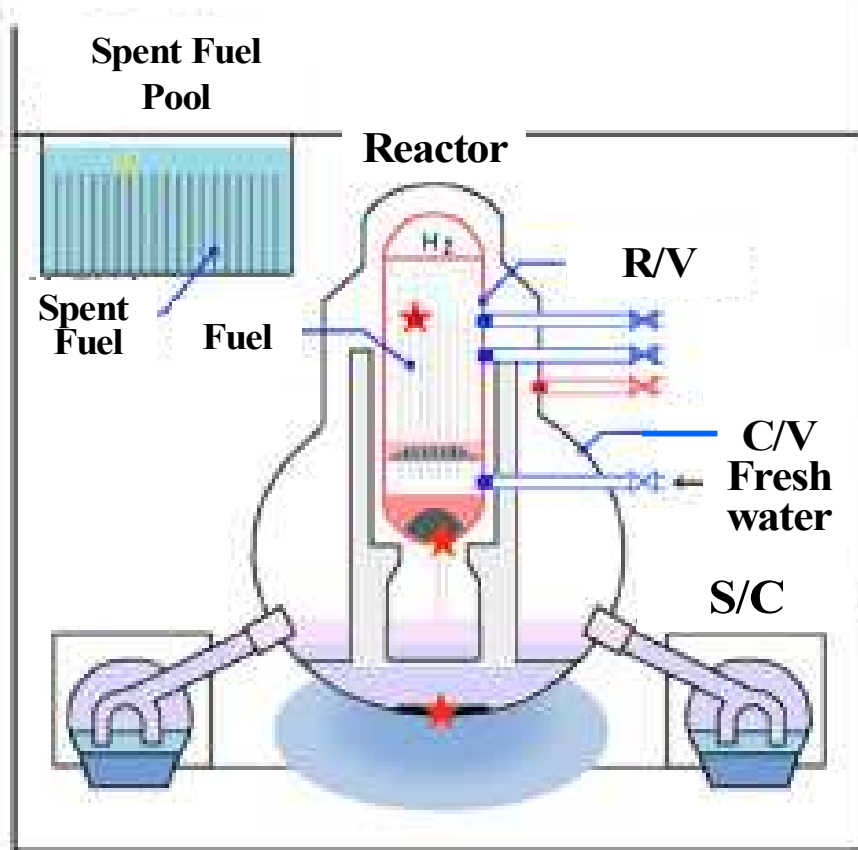


Plant Parameters	1F2
Temperature at feed water nozzle (°C)	106.8
Reactor Pressure (MPa g)	0.012
Reactor Coolant Level (mm)	-1850
Temperature in Spent Fuel Pool (°C)	34.0

source : [www.tepco.co.jp](http://www.tepco.co.jp) and [fukumitsu.xii.jp/syu\\_f/FukushimaGenpatsu\\_1.html](http://fukumitsu.xii.jp/syu_f/FukushimaGenpatsu_1.html)

# Status of 1F3 As of 24 August 2011

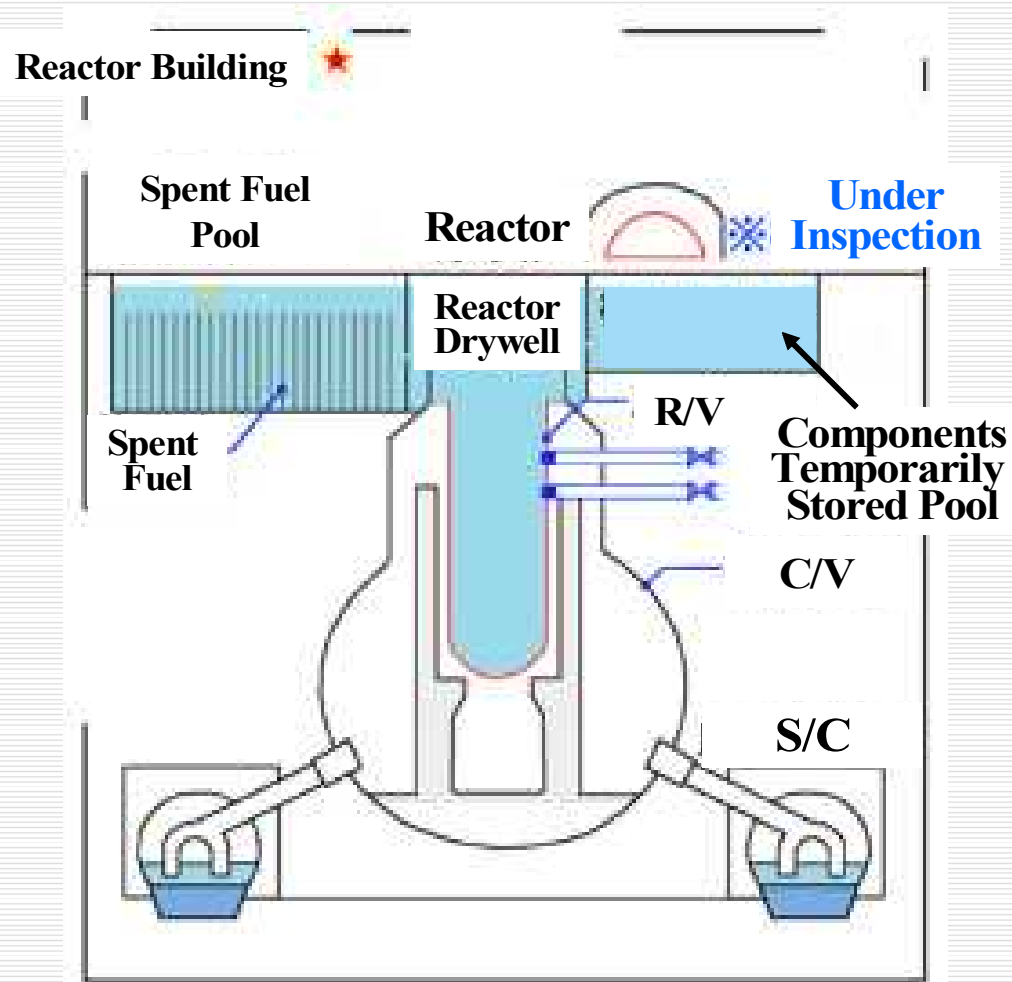
Reactor Building ★



<b>Plant Parameters</b>	<b>1F3</b>
<b>Temperature at feed water nozzle (°C)</b>	<b>112.0</b>
<b>Reactor Pressure (MPa g)</b>	<b>-0.183</b>
<b>Reactor Coolant Level (mm)</b>	<b>-1750</b>
<b>Temperature in Spent Fuel Pool (°C)</b>	<b>31.6</b>

source : [www.tepco.co.jp](http://www.tepco.co.jp) and [fukumitsu.xii.jp/syu\\_f/FukushimaGenpatsu\\_1.html](http://fukumitsu.xii.jp/syu_f/FukushimaGenpatsu_1.html)

# Status of 1F4 As of 24 August 2011



Plant Parameters	1F4
Temperature at feed water nozzle (°C)	-
Reactor Pressure (MPa)	-
Reactor Coolant Level (mm)	-
Temperature in Spent Fuel Pool (°C)	40

source : [www.tepco.co.jp](http://www.tepco.co.jp) and [fukumitsu.xii.jp/syu\\_f/FukushimaGenpatsu\\_1.html](http://fukumitsu.xii.jp/syu_f/FukushimaGenpatsu_1.html)

# Treatment of Contaminated Water

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- A large amount of fresh and sea water were discharged into reactor cores of 1F1-3, and SFP of 1F4.
- These water contaminated by radioactive materials were free to diffuse from R/C or SFP to R/B.
- Contaminated water existed in R/B was recovered into special tank and **a fraction of it was leaked from R/B into sea water.**
- **Very low activated water was released from the tank to sea.**
- Some of it is recycled for cooling R/C and SFP by use of newly equipped system. Then, cooling system is changed from once through system to closed recycle system.

# Release of Radioactive Materials to the Sea

Events	Results
Leakage from 1F2 (Apr. 2 - Apr. 6)	<p>On April 2, it was discovered that <b>highly contaminated water</b> was flowing into the sea water through the crack on the lateral surface of the pit.</p> <p>Total discharged amount of the radioactive was assumed to be approximately <b><math>4.7 \times 10^{15}</math> Bq.</b></p>
Discharge to the sea (Apr. 4 - Apr. 10)	<p>In order to secure capability for highly contaminated water, TEPCO discharged <b>low level radioactive water</b> into sea water.</p> <p>Total discharged amount was presumed to be approximately <b><math>1.5 \times 10^{11}</math> Bq.</b></p>
Leakage from 1F3 (May 11)	<p>On May 11, TEPCO confirmed the outflow from a pit near Channel of 1F3 into the sea.</p> <p>Total amount can be estimated to be <b><math>2.0 \times 10^{13}</math> Bq.</b></p>

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety

# Amount of Radioactive Materials Discharged

	<b>I-131</b>	<b>Cs-137</b>
<b>Amount of RM discharged to the atmosphere (Bq)</b>	<b><math>1.6 \times 10^{17}</math></b>	<b><math>1.5 \times 10^6</math></b>

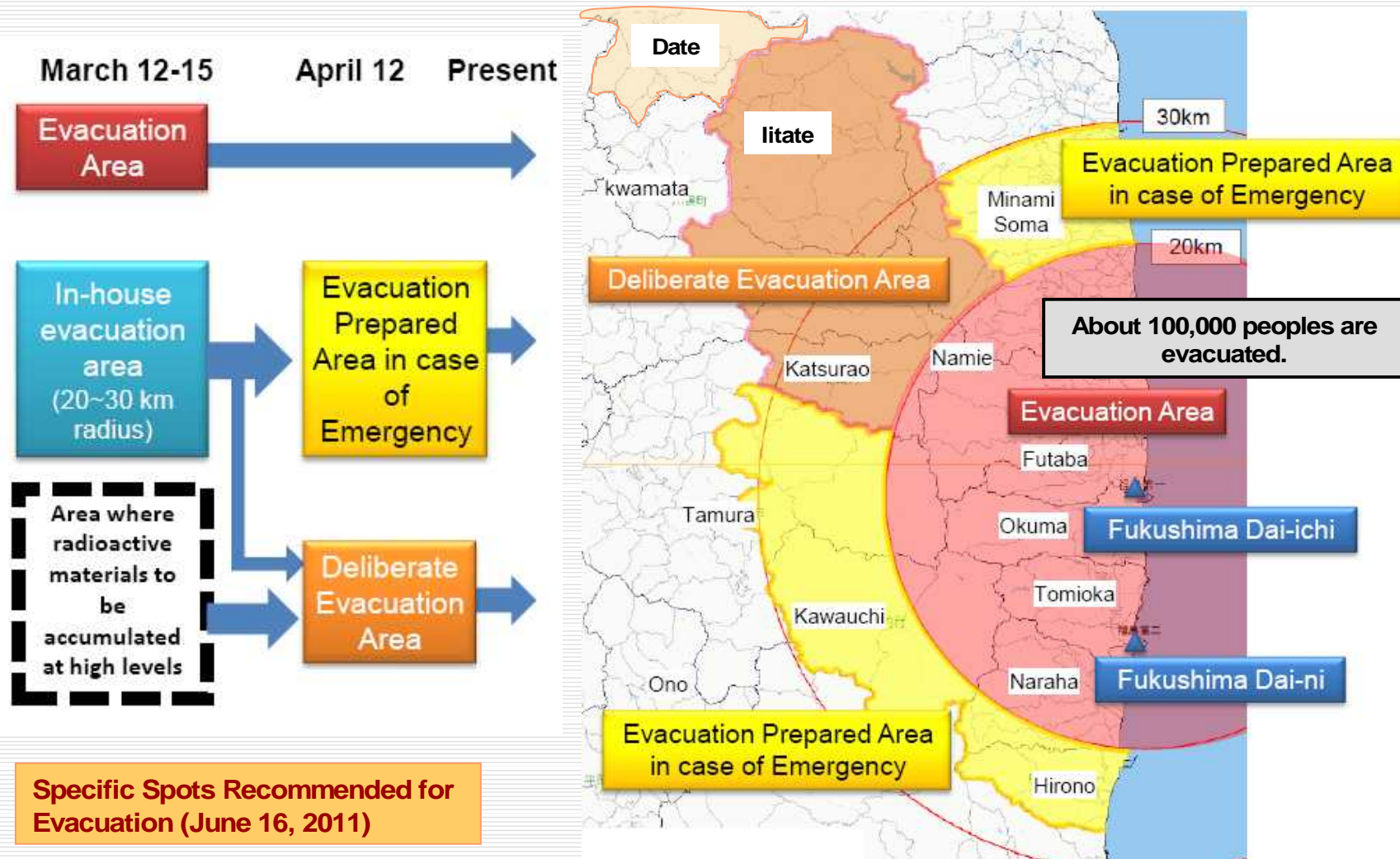
	<b>Noble gases</b>	<b>Iodine</b>	<b>Other nuclides</b>
<b>Release Rate (%) at 1F1</b>	<b>100</b>	<b>1</b>	<b>less than 1</b>
<b>Release Rate (%) at 1F2</b>	<b>100</b>	<b>0.4-7</b>	<b>Te:0.4-3 Ce:0.3-6</b>
<b>Release Rate (%) at 1F3</b>	<b>100</b>	<b>0.4-0.8</b>	<b>0.3-0.6</b>

analyzed by use of MELCOR code

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety



# Evacuation of Neighborhoods



source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety

# INES Rating

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NISA issued provisional INES ratings, based on “What is known” at the time.

<b>Timing</b>	<b>INES Ratings</b>	<b>Criteria</b>
<b>March 11</b>	<b>Level 3 for 1F1, 2 and 3 Level 3 for 2F1, 2 and 4</b>	<b>Defense in Depth</b>
<b>March 12</b>	<b>Level 4 for 1F1</b>	<b>Radiological Barriers and Control</b>
<b>March 18</b>	<b>Level 5 for 1F1, 2 and 3 Level 3 for 1F4</b>	<b>Radiological Barriers and Control Defense in Depth</b>
<b>April 12</b>	<b>Level 7 for 1F NPS</b>	<b>People and Environment</b>

**Official rating will be done after cause and countermeasures are identified.**

[source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety](#)

# Current Status of Nuclear Energy Policy

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- Chubu EPC is requested to stop all Hamaoka NPPs due to political reason.
- It is proposed that renewable energies should be developed instead of nuclear energy.
- Restart of NPPs whose periodic inspection has been finished or will be started is uncertain though two step stress test is scheduled on NPPs.
- If this situation is continued, all NPPs will be stopped.
- Development of nuclear fuel cycle including FBR is also uncertain.
- Political parties and politicians in Japan have different policies on utilization nuclear energy in future.

source :news paper and so on

# Current Status of NPP in Japan As of 25 August

EC	Plant	Status	EC	Plant	Status	EC	Plant	Status
Hokkaido EPCO	Tomari-1	PI	TEPCO	K-1	PI	KEPCO	Takahama-1	PI
	Tomari-2	PI		K-2	SD		Takahama-2	OP
	Tomari-3	OP		K-3	SD		Takahama-3	OP
Tohoku EPCO	Higashi-dori	SD		K-4	SD		Takahama-4	PI
	Onagawa-1	SD		Chugoku EPCO	K-5	OP	Shimane-1	PI
	Onagawa-2	SD			K-6	OP	Shimane-2	OP
Hokuriku EPCO	Onagawa-3	SD		Chubu EPCO	K-7	PI	Shikoku EPCO	Ikata-1
	Shika-1	SD	Hamaoka-1		DC	Ikata-2		OP
TEPCO	Shika-2	PI	Hamaoka-2		DC	Ikata-3		PI
	1F-1	SD	Hamaoka-3		PI	Kyushu EPCO	Genkai-1	OP
	1F-2	SD	Hamaoka-4		SD		Genkai-2	PI
	1F-3	SD	Hamaoka-5	SD	Genkai-3		PI	
	1F-4	SD	KEPCO	Mihama-1	PI		Genkai-4	OP
	1F-5	SD		Mihama-2	OP	Sendai-1	PI	
	1F-6	SD		Mihama-3	PI	Sendai-2	OP	
	2F-1	SD		Ooi-1	PI	JAPC	Tokai-1	DC
	2F-2	SD	Ooi-2	OP	Tokai-2		PI	
2F-3	SD	Ooi-3	PI	Tsuruga-1	PI			
2F-4	SD	Ooi-4	PI	Tsuruga-2	SD			

OP: Operation (13) PI: Periodic Inspection (20) SD: Shutdown (21) DC: Decommissioning (3)

# Lessons Learned From 1F Accident (1)

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## 1. Appropriate DBAs

- Appropriate consideration for natural hazards by design
- Design basis tsunami height 5.7 m against 15 m of actual tsunami height

## 2. Robustness in responding to BDBAs such as SBO for long duration and LOHS

- Appropriate design philosophy to sustain safety function against common cause failures brought by natural hazards
  - Only 1 air cooled DG, which is located on the ground level, was survived
  - All the sea water pumps were located slightly above the design tsunami height and they were with no protection against water
- Appropriate AM measures for both prevention and mitigation of severe accidents
  - No AMs for SFP cooling and H<sub>2</sub> control in the R/B
  - No AMs training under severe conditions for multi-units under continuous aftershocks

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety

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# Lessons Learned From 1F Accident (2)

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## **3. Difficult situation for Post Severe Accident Recovery**

- **Warning for aftershocks and subsequent tsunami**
- **High radiation in working area**
- **Massive radioactive debris everywhere within the site**

## **4. Emergency Preparedness and Responses**

- **Evacuation zone**
- **Function of off-site center**
- **Communication**
- **Radiation monitoring**

# Statement of AEC on July 19, 2011

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- **Atomic Energy Commission in Japan (AEC) found out the following facts from 1F NPP accident.**
  - **To arise skepticism toward adequacy of design basis tsunami height and DBEs**
  - **To know inadequacy in defense in depth approaches**
  - **To occur severe accident whose scale is much bigger than those expected in nuclear policy**
  - **To find out problems in crisis-control structure in and out of NPP**
  
- **AEC takes the following actions.**
  - **To ask for strengthening safety measures of NPP**
  - **To set up basic strategy for future nuclear energy research, development and utilization within 2011 JFY based on atomic energy basic law**

**Original statement of AEC (Fundamental Policy Concerning Budget Estimation of Nuclear in JFY of 2012) was published in Japanese and translated into English by JNES.**

source : [www.aec.go.jp](http://www.aec.go.jp)

# Accident in Fukushima Daiichi NPP

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**Thank you very much for your attention!**



**Support Documentation**

# **Accident in Fukushima Daiichi NPP**

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**for an information purpose only**

**Dr. N. Nakae**  
**JNES**

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- 8. Careful Thinking of Nuclear Energy**

# Radiation Exposure (engaged person)

Dose limit	100 mSv → 250 mSv		
	March	April	May
Total number of engaged persons	3,538	3,254	4,772
Average radiation dose (mSv)	23.1	4.2	1.85
Number of the persons whose dose exceeds 100 mSv	111	0	0
Number of the persons whose dose exceeds 250 mSv	6	0	0

source : [www.tepco.co.jp](http://www.tepco.co.jp)

# Radiation Exposure (peripheral people)

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<b>Radiation dose level</b> <b>No. of screening people</b> (As of 21 August 2011)	<b>no harmful effect</b> <b>219,743 persons</b>
<b>Radiation dose level on childhood thyroid</b> <b>No. of screening infants</b> (As of 30 March 2011)	<b>no harmful effect</b> <b>1080</b>

source :[wwwcms.pref.fukushima.jp](http://wwwcms.pref.fukushima.jp)  
[www.nsc.go.jp](http://www.nsc.go.jp)

# Amount of Radioactive Materials Discharged

	Fukushima Daiichi NPS		Chernobyl	TMI
	NISA	NSC		
I-131 (a)	$1.3 \times 10^{17}$	$1.5 \times 10^{17}$	$1.8 \times 10^{18}$	$5.6 \times 10^{11}$
Cs-137	$6.1 \times 10^{15}$	$1.2 \times 10^{16}$	$8.5 \times 10^{16}$	negligible
Reduced Iodine (b)	$2.4 \times 10^{17}$	$4.8 \times 10^{17}$	$3.4 \times 10^{18}$	negligible
(a) + (b)	$3.7 \times 10^{17}$	$6.3 \times 10^{17}$	$5.2 \times 10^{18}$	$5.6 \times 10^{11}$

source : Report of the Japanese Government to IAEA Ministerial Conference on Nuclear Safety  
 T. Watarai, Y. Inoue, F. Masuda, J. of the Atomic Energy Society of Japan, Vol.32, No.4 (1990)

# Integrated Dose at Reading Points



source: <http://radioactivity.mext.go.jp/>

# Road Map of Recovery Plan

Current Status of "Roadmap towards Restoration from the Accident at Fukushima Daiichi Nuclear Power Station, TEPCO" (Revised edition)

July 19, 2011  
 Nuclear Emergency Response Headquarters  
 Government-TEPCO Integrated Response Office

Appendix 1-③

Red colored: newly added to the previous version, ☆: already reported to the government

Issues	As of April 17	Step 1 (around 3 months)	Step 2 (around 3 to 6 months after achieving Step 1) current status (as of July 17)	Mid-term issues (around 3 years)
I. Cooling	(一) Reactor Fresh water injection	Cooling by minimum injection rate (injection cooling) Consideration and preparation of reuse of accumulated water	Circulating Injection Cooling (start) ☆ Stable cooling	Cold shutdown condition Continuous cold shutdown condition Protection against corrosion cracking of structural materials* *partially ahead of schedule
		Nitrogen gas injection ☆ Improvement of work environment ☆	Circulating Injection Cooling (continued)	
	(二) Spent Fuel Pool Fresh water injection	Reliability improvement in injection operation / remote-control operation *ahead of schedule Circulation cooling system ☆ (installation of heat exchanger) *partially ahead of schedule	Stable cooling Remote-controlled injection operation Consideration / installation of heat exchanging function	More stable cooling Start of removal work of fuels
II. Mitigation	(三) Accumulated Water Transferring water with high radiation level Storing water with low radiation level	Installation of storage / processing facilities ☆	Secure storage place Expansion / consideration of full-fledged processing facilities Decontamination ☆ / desalt processing (reuse), etc Storage ☆ / management of sludge waste etc. Mitigation of contamination in the ocean	Reduction of total amount of contaminated water Installation of full-fledged water processing facilities Continuous processing of accumulated water Research of processing of sludge waste etc. Mitigation of contamination in the ocean
		Installation of storage facilities / decontamination processing		
	(四) Ground water Mitigation of contamination of groundwater Consideration of method of shielding wall of groundwater	Mitigate ocean contamination (continued) Design / start of implementation of shielding wall of groundwater	Mitigate ocean contamination (continued) Solidification of contaminated soil, etc Establishment of shielding wall of groundwater	
	(五) Atmosphere / Soil Dispersion of inhibitor Removal of debris	Mitigate scattering Installation of reactor building cover (Unit 1) ☆ Removal of debris (top of Unit 3&4 R/B) Consideration of reactor building container	Mitigate scattering (continued) Removal of debris / installation of reactor building cover (Unit 3&4) Start of installation work of reactor building container	

# Emergency Safety Precaution

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- Nuclear and Industrial Safety Agency (NISA) called for the emergency safety precaution on 30 March, 2011.
- The emergency safety caution consists of the following six items.

**Implementing the emergency inspections of equipments and facilities** to ensure the readiness for tsunami induced emergencies.

**Implementing a review of the Emergency Preparedness Plan and conducting drills** with the assumption that all alternating current power sources, seawater cooling function and Spent Fuel Pool cooling function have been lost.

**Ensuring the alternative power sources** that can supply necessary power in a timely manner when both on-site power and emergency power supply are lost.

**Preparing for the measures to recover heat removal function in a timely manner** with the assumption of loss of seawater system facility or its function.

**Implementing the measures to supply coolant water to Spent Fuel Pools in timely manner** when cooling function for the pools and usual on-site water supply to the pools are lost.

**Implementing necessary measures taking into account the structural configuration of each NPS site**

source :News Release from NISA on March 30, 2011

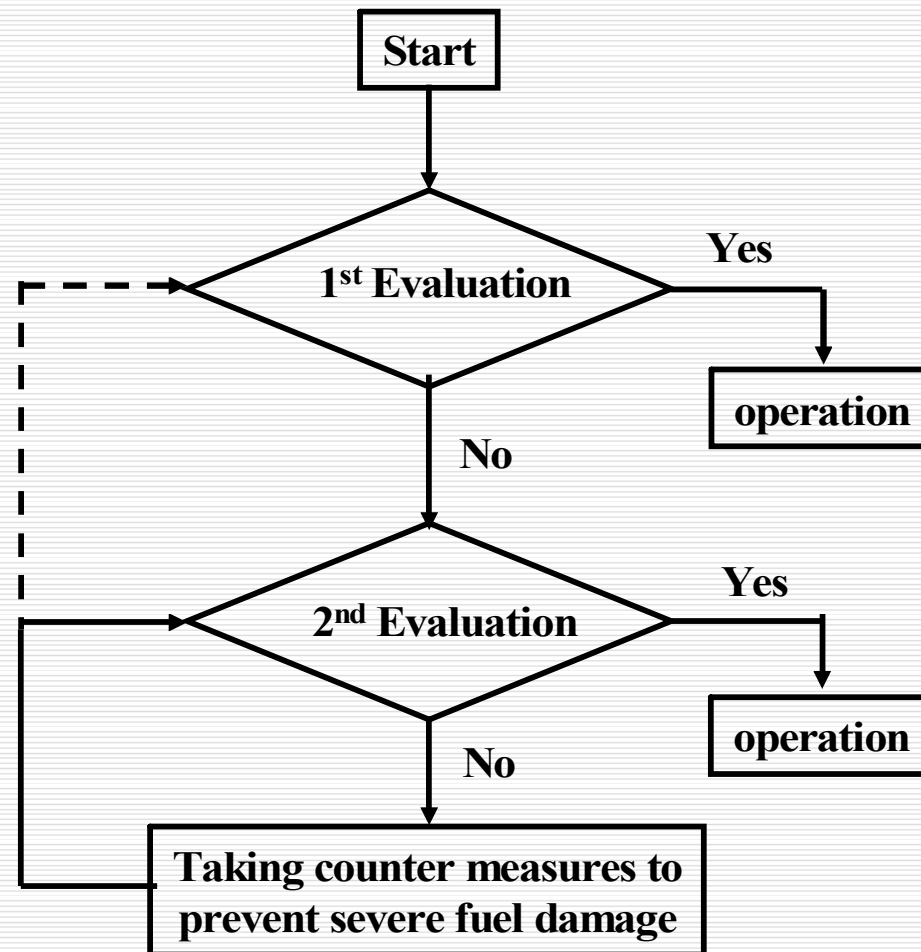


# Conducting Stress Tests

	First Round Evaluation	Second Round Evaluation
<b>Intended Events</b>	<b>Natural Hazard</b> Earthquake, Tsunami, Overlapping <b>Loss of Safety Functions</b> Electric Power Supply, Final Heat Sink, <b>Severe Accident Management</b>	<b>Natural Hazard</b> Earthquake, Tsunami, Overlapping <b>Loss of Safety Functions</b> Electric Power Supply, Final Heat Sink, <b>Overlapping</b> <b>Severe Accident Management</b>
<b>Contents</b>	To evaluate safety margin for hypothetical events (beyond design basis accident, BDBA) <b>To check if evaluation results are satisfied with acceptable values</b> <b>To confirm that NPP has a certain level of safety margin</b>	To evaluate safety margin for hypothetical events (beyond design basis accident, BDBA) <b>To identify cliff edge for severe damage of fuel</b> <b>To examine measures to prevent severe fuel damage</b>
<b>Remarks</b>	<b>Japan Original</b> <b>Apply to NPP under inspection</b>	<b>Correspond to European Stress Test</b> <b>Apply to NPP under operation</b>

source : [www.meti.go.jp](http://www.meti.go.jp)

# Conducting Stress Tests (Flow Sheet)



source :N. Nakae, private communication

# Careful Thinking of Nuclear Energy

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- **Processing the Accident of Fukushima Daiichi Nuclear Power Plants**
- **Safety Improvement of Existing Nuclear Power Plants**
- **Procession of the Development of Nuclear Fuel Cycle including LMFBR**

# Processing the Accident of 1F NPPs

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## **(1) Radiation Exposure of General Public living in the Vicinity**

**Careful Follow-up of the Effect on Health of General Public Received Radiation Dose**

**Designing Criteria of Dose Limit for Severe Accident in Order to Control Radiation Exposure of the Workers and the People in the Surrounding Area Based on the Reporting by ICRP**

**Explanation of the Background of the Criteria done by the Government**

## **(2) Recovery of Environment Contamination**

**Assurance of the Period of Evacuation and Safety of the Environment when they come back again**

**Returning Evacuated Peoples Back to Original Places as soon as possible**

**Cleaning Contaminated Soil, Building, Plants and others up**

## **(3) Treatment and Disposal of NPPs Damaged**

**Handling and Reprocessing of Fuels Stored in Damaged Core and SFP**

**Decommissioning Damaged NPPs Including Environment Clean-up**

[source :N. Nakae, private communication](#)

# Safety Improvement of Existing NPPs

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- **Review of Guides** of Safety Design, Safety Assessment, Site Evaluating, Severe Accident (SA), its related Accident Management (AM), Radiation Dose Limit, and Evacuation
- **Review of Safety Design** such as Water Proof of Components, Electric Power Supply, Multiple and Multiplicity of Safety Protection System, Monitoring of Plant Parameters, Seismic, and Tsunami
- **Review of Safety Assessment** such as External Events, especially Natural Hazard such as Earthquake and Tsunami
- **Review of Site Evaluating** in relation with Severe Accident (SA) in view point of Determining Source Term and Method of Evaluating Dose Effect on the Public and Environment
- **Extraction of Systematic Fuel Failure Modes** in SA and Taking into Account of Corresponding AM
- **Reflection of the Result of Reviewing AM and Stress Test** in Education and Training of Operators also in Checking and Testing Safety Protection System

source :N. Nakae, private communication

# Procession of the Development of NFC

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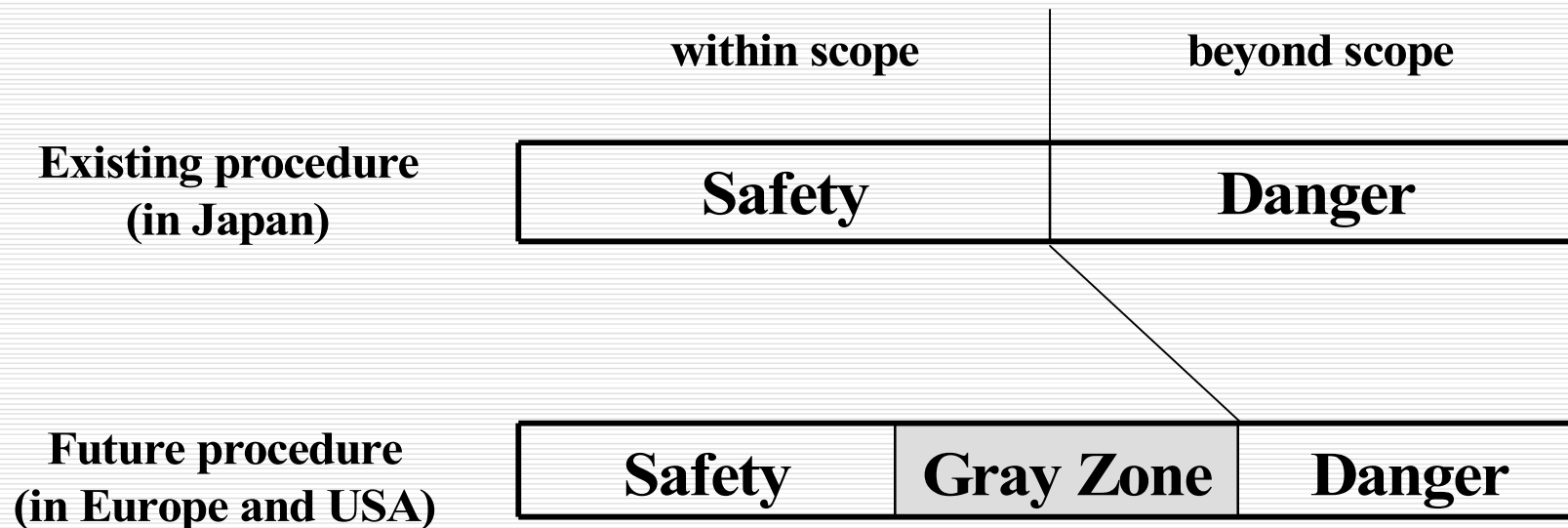
- **Technology of Reprocessing** Contributes to Handling and Treatment of Fuels in Damaged Core and Spent fuel Pool
- **Technology of High Level Waste Disposal** Contributes to Recovery of Environment Contamination
- **Improvement of Safety of LMFBR** contributes to strengthening of Safety of Existing Nuclear Power Plants
- **Achievement of Nuclear Fuel Cycle Including LMFBR** Contributes to Realizing Sustainable Social World of Friendly Earth Environment

source :N. Nakae, private communication

# Background

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**An approach taking into account for concept of risk shall be needed in safety licensing of nuclear facilities.**



# Why Needs To Introduce Concept of Risk?

Gray zone might exist in safety licensing of nuclear facilities.

